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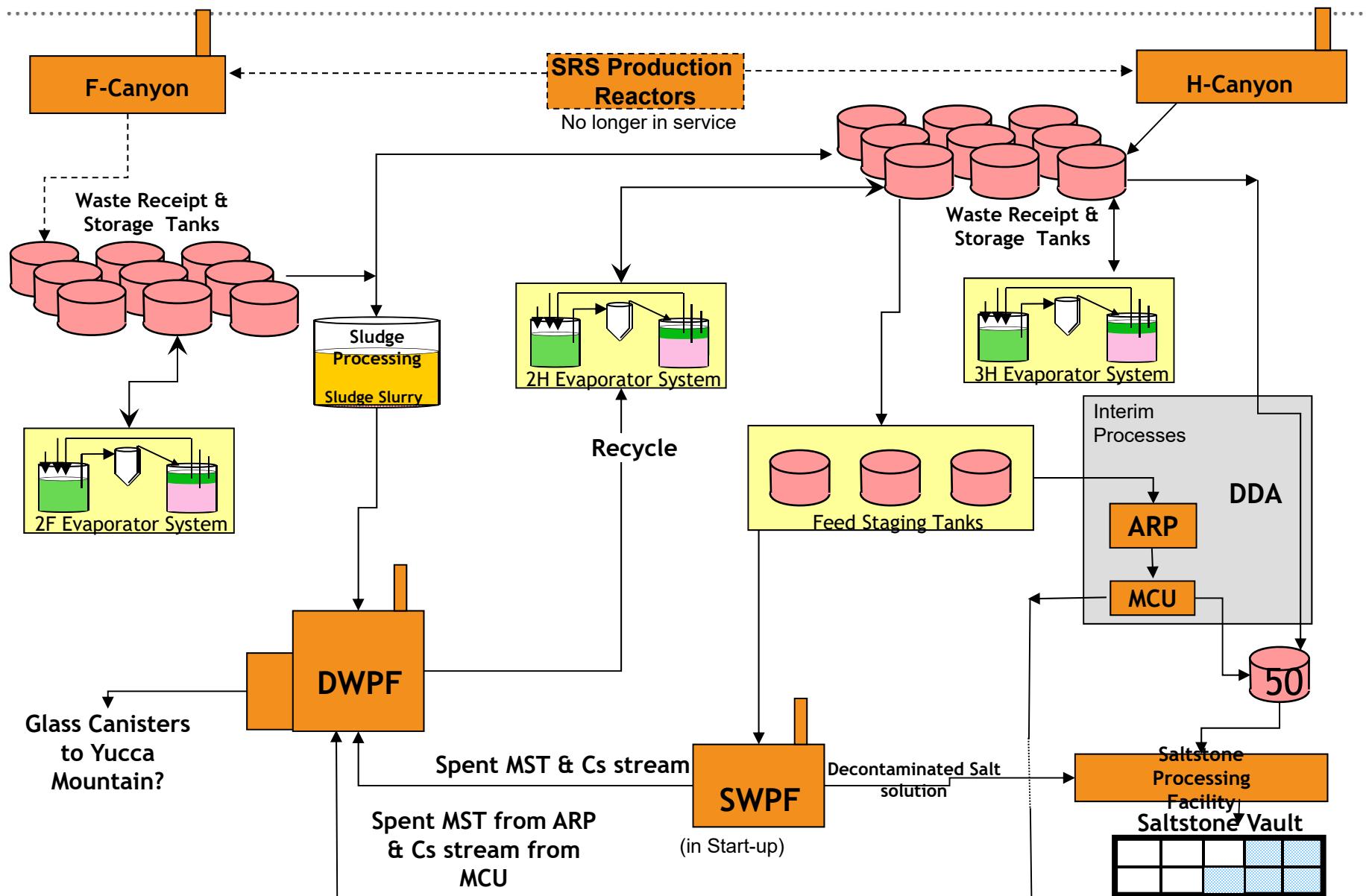


Applications of Eichrom Resins to Savannah River Site Highly Radioactive Sample Matrices

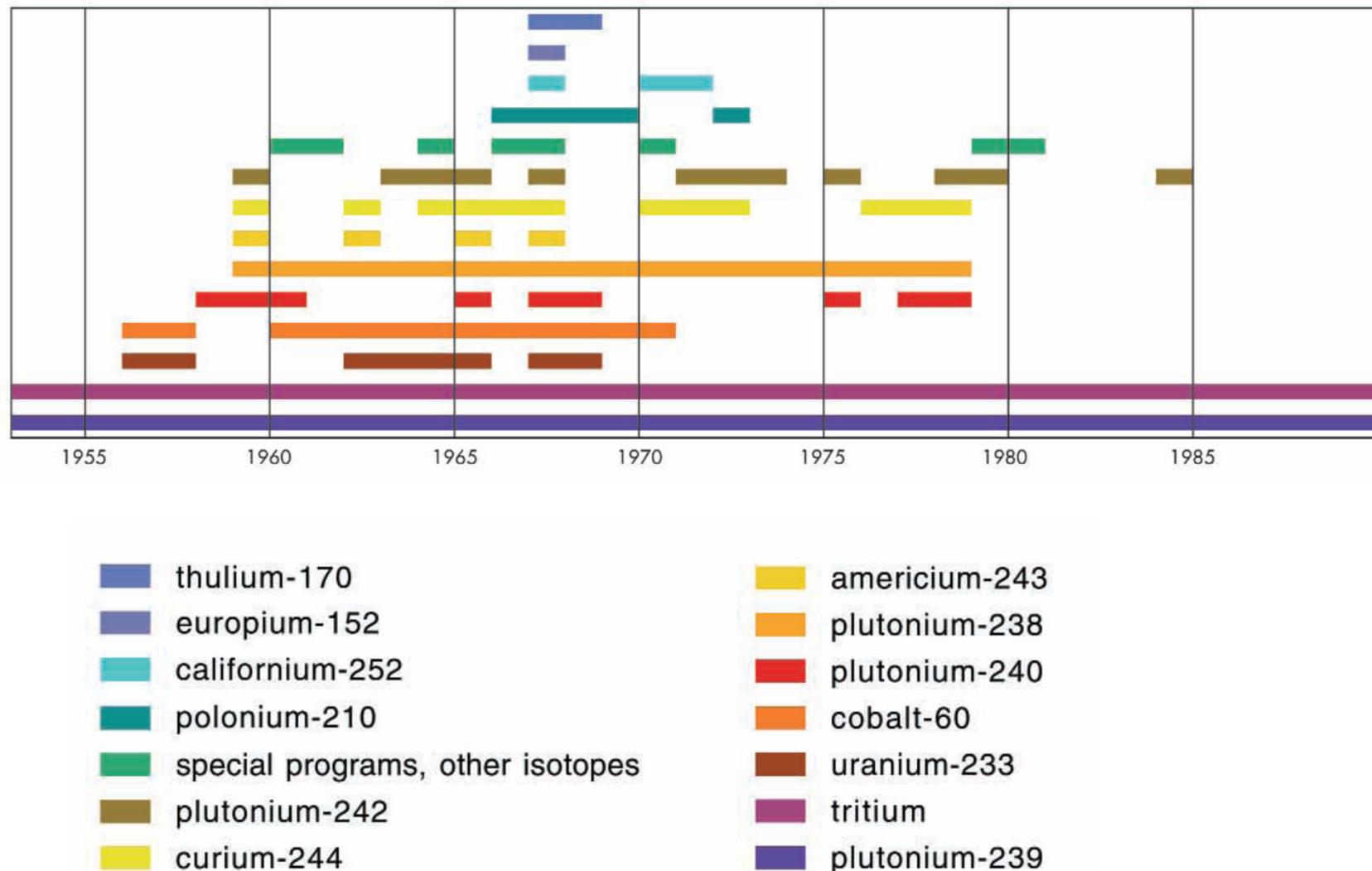
David DiPrete
Nuclear Measurements Group/Analytical Development Section

Eichrom UGM RRMC 2019

Savannah River Site High Level Waste Flowsheet



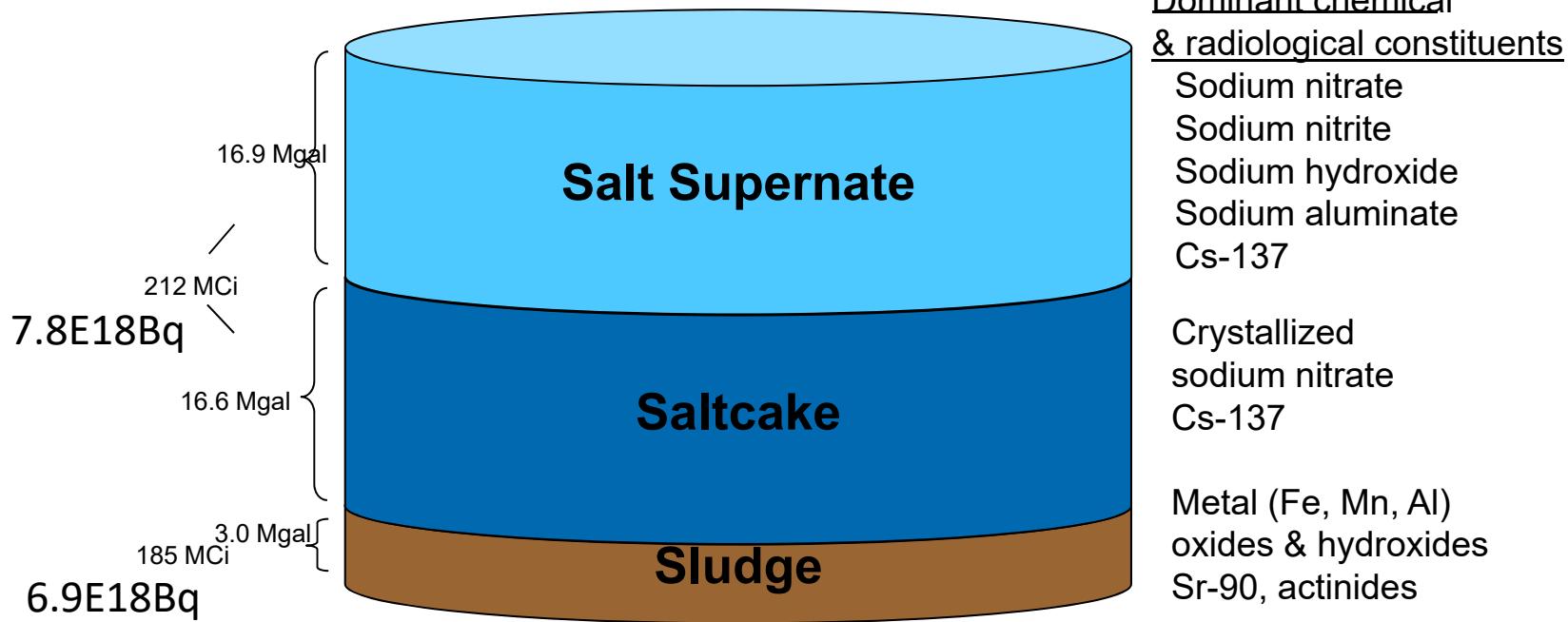
Radionuclides Produced at SRS



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F and H Tank Farms



Two tank farms

49 waste tanks

- 22 “old-style” tanks
- 27 “new-style” tanks

Approximately 37 million gallons of waste



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Nuclear Measurements Group

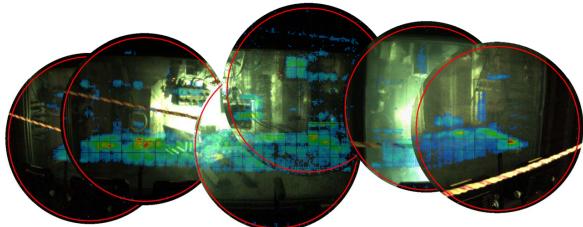
STAFFING

**4 PhD Nuclear Chemists 2 PhD Nuclear Engineers
4 BS Chemists 1 BS Physicist
2 Specialists 1 Laboratory Technician**

Customized Radiochemistry

H-3	C-14	Al-26	Cl-36	K-40	Ca-41	Ni-59	Co-60	Ni-63	Se-79	Rb-87
Sr-90	Y-90	Mo-93m	Nb-93m	Zr-93	Nb-94	Zr-95	Tc-99	Ru-103	Ru-106	Pd-107
Ag-108m	Ag-110m	Sn-121m	Sb-125	Te-125m	Sb-126	Sn-126	I-129	I-131	Ba-133	Cs-134
Cs-135	Ba-137m	Cs-137	Ba-140	Ce-141	Ce-144	Nd-147	Pm-147	Sm-151	Eu-152	Eu-154
Eu-155	Pt-193m	Bi-207	Tl-208	Bi-210	Pb-210	Pb-210	Po-210	Po-210	Bi-212	Pb-212
Po-212	Bi-214	Pb-214	Po-214	Po-216	Po-218	Rn-220	Rn-222	Ra-224	Ra-226	Ac-227
Ac-228	Ra-228	Th-228	Th-229	Th-230	Pa-231	Th-231	Th-232	U-232	U-233	Pa-234
Th-234	U-234	U-235	U-236	Np-237	Pu-238	U-238	Pu-239	Pu-240	Am-241	Pu-241
Am-242m	Cm-242	Pu-242	Am-243	Cm-243	Cm-244	Pu-244	Cm-245	Cm-246	Bk-247	Cm-247
			Cm-248	Bk-249	Cf-249	Cf-250	Cm-250	Cf-251	Cf-252	

Non-Destructive Assay



Gamma Imaging

Neutron Multiplicity Counting

Field Radiological Assays

Calorimetry

Instrument Development



University Collaborations

Energy Frontier Research Center



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Nuclear Measurements Group - Radiochemistry

H-3	C-14	Al-26	Cl-36	K-40	Ca-41	Ni-59	Co-60	Ni-63	Se-79	Rb-87
Sr-90	Y-90	Mo-93m	Nb-93m	Zr-93	Nb-94	Zr-95	Tc-99	Ru-103	Ru-106	Pd-107
Ag-108m	Ag-110m	Sn-121m	Sb-125	Te-125m	Sb-126	Sn-126	I-129	I-131	Ba-133	Cs-134
Cs-135	Ba-137m	Cs-137	Ba-140	Ce-141	Ce-144	Nd-147	Pm-147	Sm-151	Eu-152	Eu-154
Eu-155	Pt-193m	Bi-207	Tl-208	Bi-210	Pb-210	Pb-210	Po-210	Po-210	Bi-212	Pb-212
Po-212	Bi-214	Pb-214	Po-214	Po-216	Po-218	Rn-220	Rn-222	Ra-224	Ra-226	Ac-227
Ac-228	Ra-228	Th-228	Th-229	Th-230	Pa-231	Th-231	Th-232	U-232	U-233	Pa-234
Th-234	U-234	U-235	U-236	Np-237	Pu-238	U-238	Pu-239	Pu-240	Am-241	Pu-241
Am-242m	Cm-242	Pu-242	Am-243	Cm-243	Cm-244	Pu-244	Cm-245	Cm-246	Bk-247	Cm-247
		Cm-248	Bk-249	Cf-249	Cf-250	Cm-250	Cf-251	Cf-252		

- NMG develops customized radiochemical separations and analyses upon request
 - SRS conducted diverse radioisotope production campaigns,
 - The remnants of these programs remain scattered across SRS's waste tank farms, providing a constant radioanalytical challenges
 - Sample types range from
 - Various high activity waste matrices from SRS as well as other DOE Sites
 - forensic analyses i.e. FBI's NFAC samples, WIPP fire, INL drum explosion, PORTS Off-Site Contamination question,
 - environmental samples for which SRS's production environmental and off-site commercial laboratories cannot provide services
 - Particular expertise involves measuring trace radiological isotopes in the presence of 10 or more orders of magnitude of interfering isotopes
 - The Nuclear Measurements Group fills in the gap for any emergent radioanalytical request that production or commercial radioanalytical laboratories will not support



Radiochemistry Preparation Laboratories

- **5 Laboratory Modules**

- 2 Chemical Hoods
- 3 Gloveboxes
 - Ability to work with up to 400 grams Plutonium
- 10 Radiological Hoods, 3 Radiobenches
 - Routinely work with samples containing up to $1E+10$ dpm ($1.67E+8$ Bq) alpha and beta
 - Routinely work with samples having up to 10 mRem/h (0.1 mSv/h) whole body dose @ 30cm
 - Routinely work with samples having up to 2000 mRem/h (20 mSv/h) contact dose



- **SRNL Shielded Cells Facility utilized for initial separation steps of high-activity samples**



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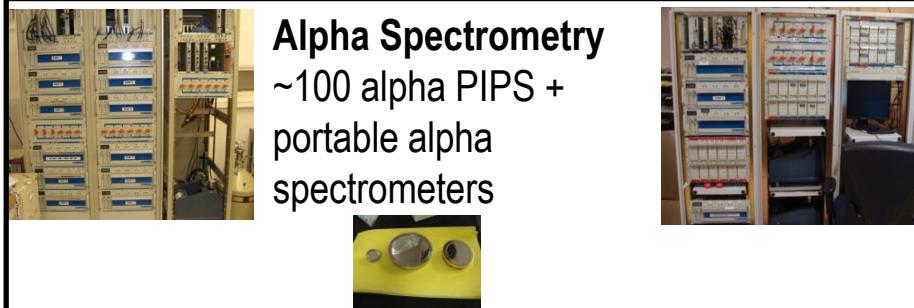
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Nuclear Measurement Counting Instrumentation

- 4 counting rooms contain nuclear measurement instrumentation
- In addition, the radiochemistry group leverages additional ADS instrumentation for radiochemical analyses
 - i.e. ICP-MS, ICP-AES



Beta Spectrometry - Triple, double, single PMT LSC counters, cosmic suppressed LSC, Portable beta spectrometers, conversion electron spectrometer, GFPCs, beta PIPS



Alpha Spectrometry
~100 alpha PIPS +
portable alpha spectrometers



Gamma Spectrometry

- 15 shielded spectrometers ranging from planar to coaxial to well HPGe
- 4 automated systems
- Numerous field deployable x-ray and gamma-ray spectrophotometers
- Calibrations generated with NIST traceable standards, Canberra LABSOCS/ISOCS, or with Customized MCNP Models

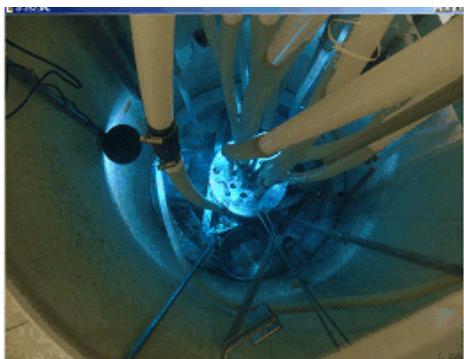
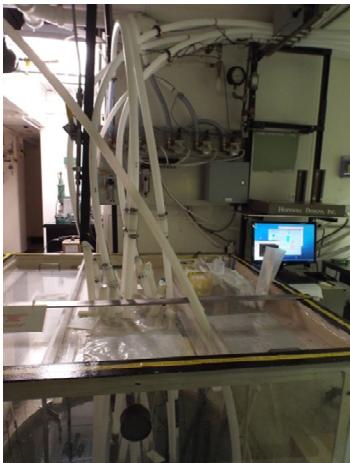


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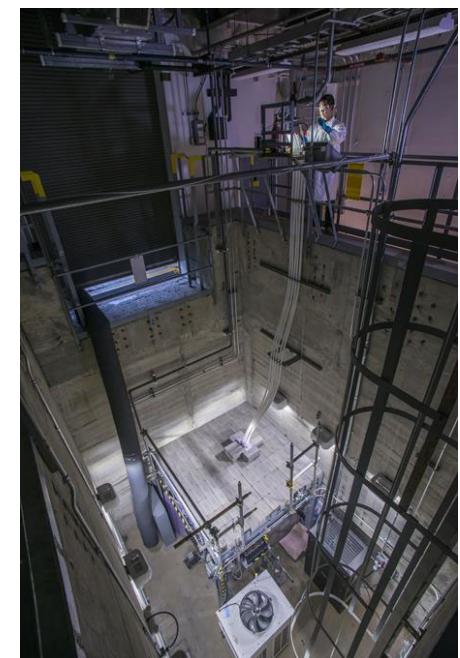
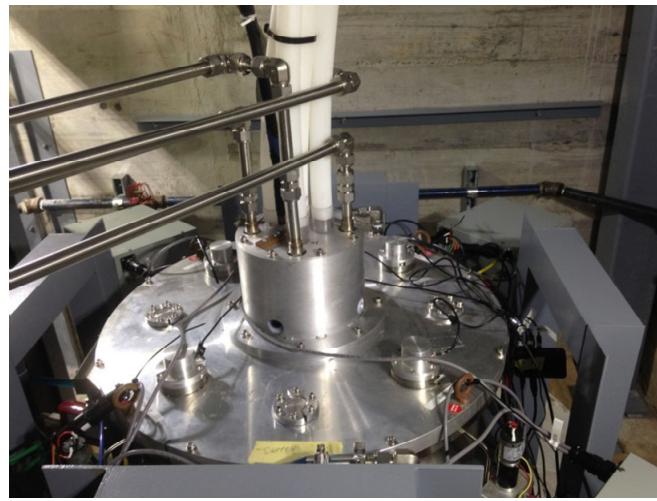
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Neutron Activation Analysis Facilities

- Supports radiological tracer production (i.e. Tc-99m)
- Supports radiochemical separation tracer recovery measurements
 - Iodine, selenium, strontium, samarium



- ~2mg (decayed from ~60mg in 2003) Cf-252 generates $\sim 1E7$ n/s/cm² thermal neutron flux
- Pneumatic system allows for repeated irradiations
- System being replaced with an Adelphi D-D neutron generator with $\sim 5E7$ n/s/cm² thermal neutron flux



Tank Closure Campaigns



Pictures of 2 SRS Waste Tanks
Following Mechanical Cleaning

H-3	C-14	Ni-59	Ni-63	Co-60	Se-79
Sr-90	Y-90	Tc-99	Sn-126	Sb-126	Cs-135
Cs-137	Ba-137m	Sm-151	Eu-152	Eu-154	Eu-155
Th-229	Th-230	U-232	U-233	U-234	U-235
U-236	U-238	Np-237	Pu-238	Pu-239	Pu-240
Pu-241	Pu-242	Pu-244	Am-241	Am-242m	Am-243
Cm-243	Cm-244	Cm-245	Cm-247	Cm-248	Cf-249
Pa-231	Ra-226	Pm-147	Ac-227	Al-26	Zr-93
Nb-94	I-129	Cl-36	K-40	Pd-107	Pt-193m

Tasked to conduct radiological characterizations on SRS Waste Tanks slated for closure

- Waste tanks slated for closure have been mechanically or chemically cleaned
- Residues are highly radioactive, as high as 1.3E9 Bq/g Beta, 1.7E7 Bq/g Alpha
- Required analyses for trace radionuclides (as low as 0.37 Bq/g) in the presence of gross levels of interfering radionuclides
- Large list of analytes requested for numerous samples of Tank Waste (up to 40 in recent campaigns)
- Cs-137 is the main contributor to whole body dose
- Sr-90/Y-90 main contributor to Extremity Dose
- Radiochemical separations run much more efficiently in radiohoods as opposed to the Shielded Cells



Tank 19 & 18 54 Radio-isotopes Requiring Characterization

H-3	C-14	Ni-59	Ni-63	Co-60	Se-79
Sr-90	Y-90	Tc-99	Sn-126	Sb-126	Cs-135
Cs-137	Ba-137m	Sm-151	Eu-152	Eu-154	Eu-155
Th-229	Th-230	U-232	U-233	U-234	U-235
U-236	U-238	Np-237	Pu-238	Pu-239	Pu-240
Pu-241	Pu-242	Pu-244	Am-241	Am-242m	Am-243
Cm-243	Cm-244	Cm-245	Cm-247	Cm-248	Cf-249
Pa-231	Ra-226	Pm-147	Ac-227	Al-26	Zr-93
Nb-94	I-129	Cl-36	K-40	Pd-107	Pt-193m

Every Waste Tank often has unique challenges even for routine analyses

Target typically to measure down to the 0.37 Bq/g neighborhood, Tank Waste in the 2.5E7 Bq/g activity range

Question becomes how many analytes will actually be present at much higher levels (makes for a much easier analysis), and how many will require procedures to get down to the 0.37 Bq/g levels, and then, can we even do it in this time frame



54 Radio-isotopes' Origins

H-3	C-14	Ni-59	Ni-63	Co-60	Se-79
Sr-90	Y-90	Tc-99	Sn-126	Sb-126	Cs-135
Cs-137	Ba-137m	Sm-151	Eu-152	Eu-154	Eu-155
Th-229	Th-230	U-232	U-233	U-234	U-235
U-236	U-238	Np-237	Pu-238	Pu-239	Pu-240
Pu-241	Pu-242	Pu-244	Am-241	Am-242m	Am-243
Cm-243	Cm-244	Cm-245	Cm-247	Cm-248	Cf-249
Pa-231	Ra-226	Pm-147	Ac-227	Al-26	Zr-93
Nb-94	I-129	Cl-36	K-40	Pd-107	Pt-193m

From Fission Products

From Activation Products

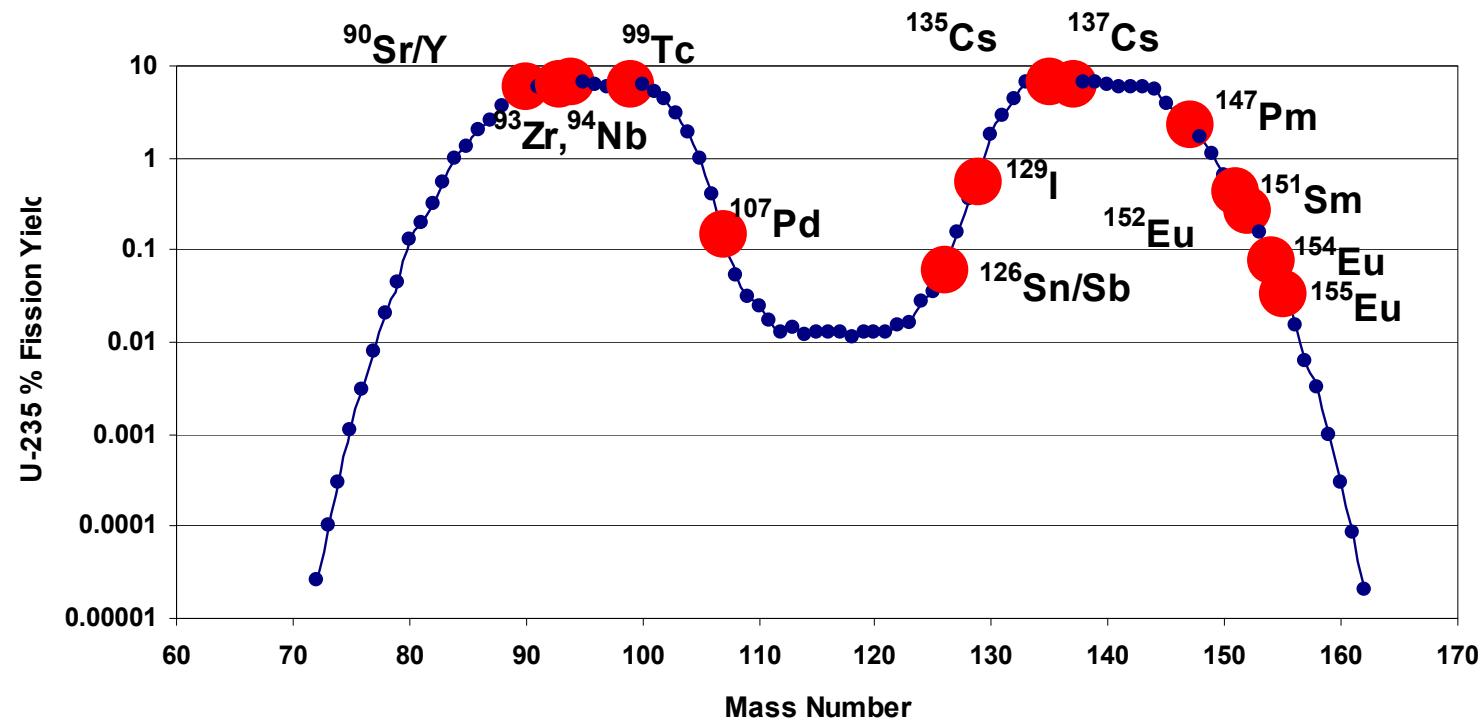
Natural



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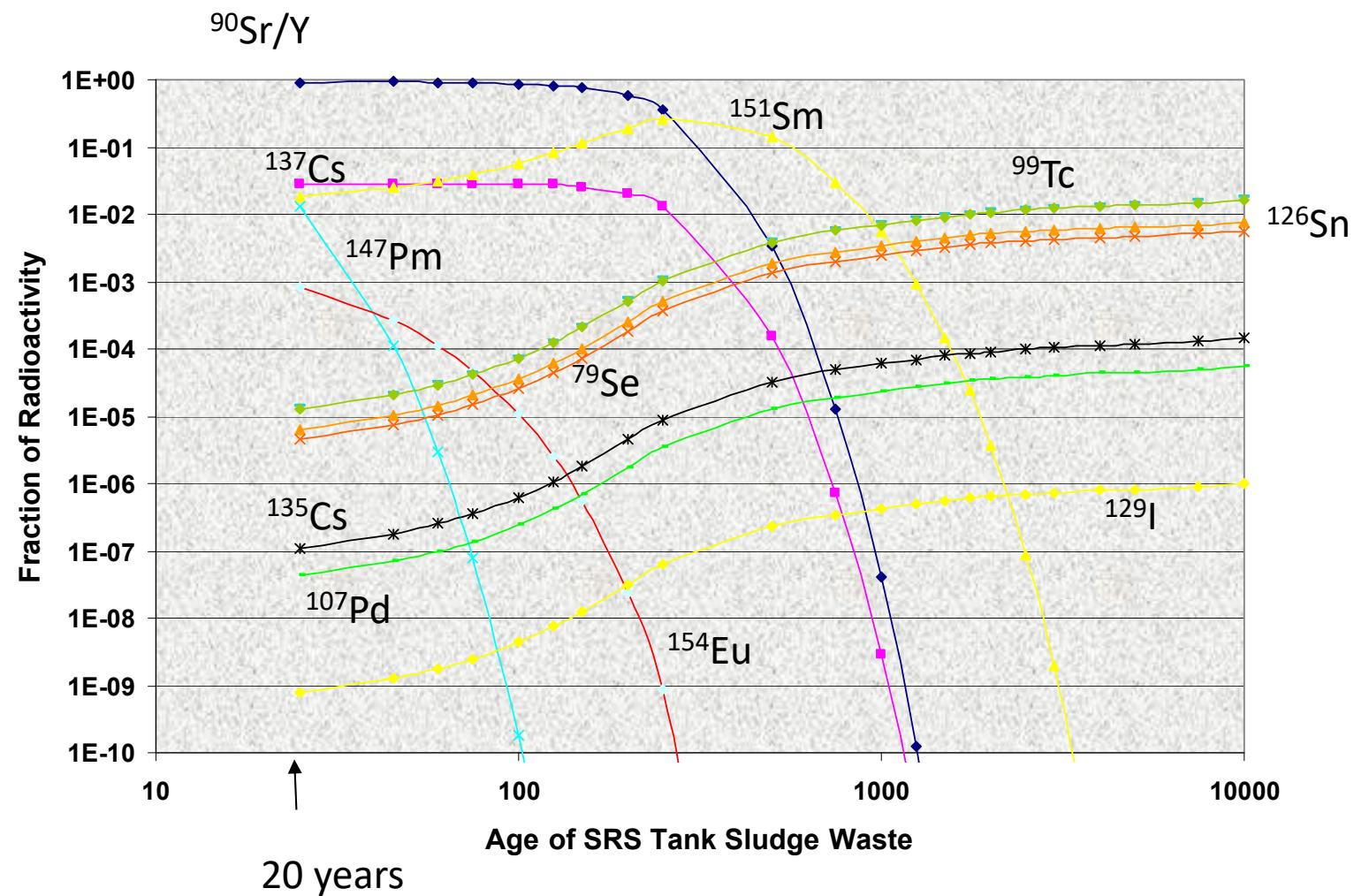
Fission Products – Fission Yield Curve



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Fission Product Distribution Over Time



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Activation Products

H-3	C-14	Ni-59	Ni-63	Co-60	
Th-229		U-232	U-233		
U-236		Np-237	Pu-238	Pu-239	Pu-240
Pu-241	Pu-242	Pu-244	Am-241	Am-242m	Am-243
Cm-243	Cm-244	Cm-245	Cm-247	Cm-248	Cf-249
				Al-26	
		Cl-36			Pt-193m

- Was the precursor present to be exposed to a neutron flux to generate levels having current activities $>0.37 \text{ Bq/g}$?
 - i.e. H-2(n,γ)H-3, or N-14(n,p)C-14 Probably so
 - i.e. Cl-35(n,γ)Cl-36 or Pt-192(n,γ)Pt-193m Probably not



Looking at Isotopes on the List That Use Eichrom Resins

Isotopes in blue make use of Eichrom Products

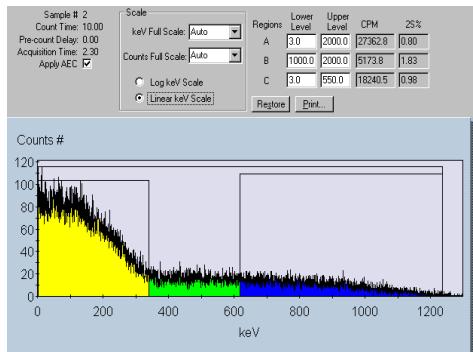
H-3	C-14	Ni-59	Ni-63	Co-60	Se-79
Sr-90	Y-90	Tc-99	Sn-126	Sb-126	Cs-135
Cs-137	Ba-137m	Sm-151	Eu-152	Eu-154	Eu-155
Th-229	Th-230	U-232	U-233	U-234	U-235
U-236	U-238	Np-237	Pu-238	Pu-239	Pu-240
Pu-241	Pu-242	Pu-244	Am-241	Am-242m	Am-243
Cm-243	Cm-244	Cm-245	Cm-247	Cm-248	Cf-249
Pa-231	Ra-226	Pm-147	Ac-227	Al-26	Zr-93
Nb-94	I-129	Cl-36	K-40	Pd-107	Pt-193m



Sr-90/Y-90

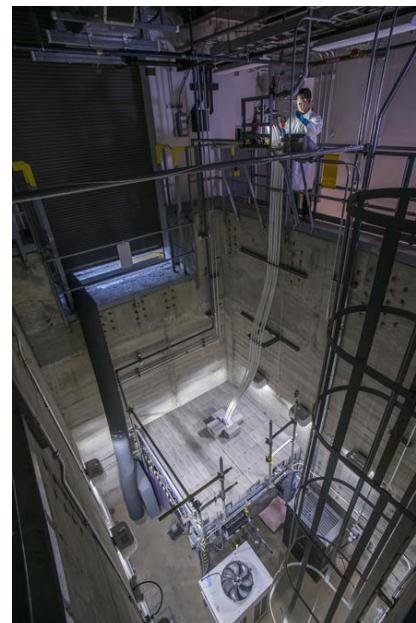
Peroxide Fusion
Digestion in Shielded
Cells

Eichrom Sr resin
Extraction



Liquid Scintillation
Counting for Sr-90

For HLW supernate we do 2
Sr extractions, for HLW
sludge we do 1



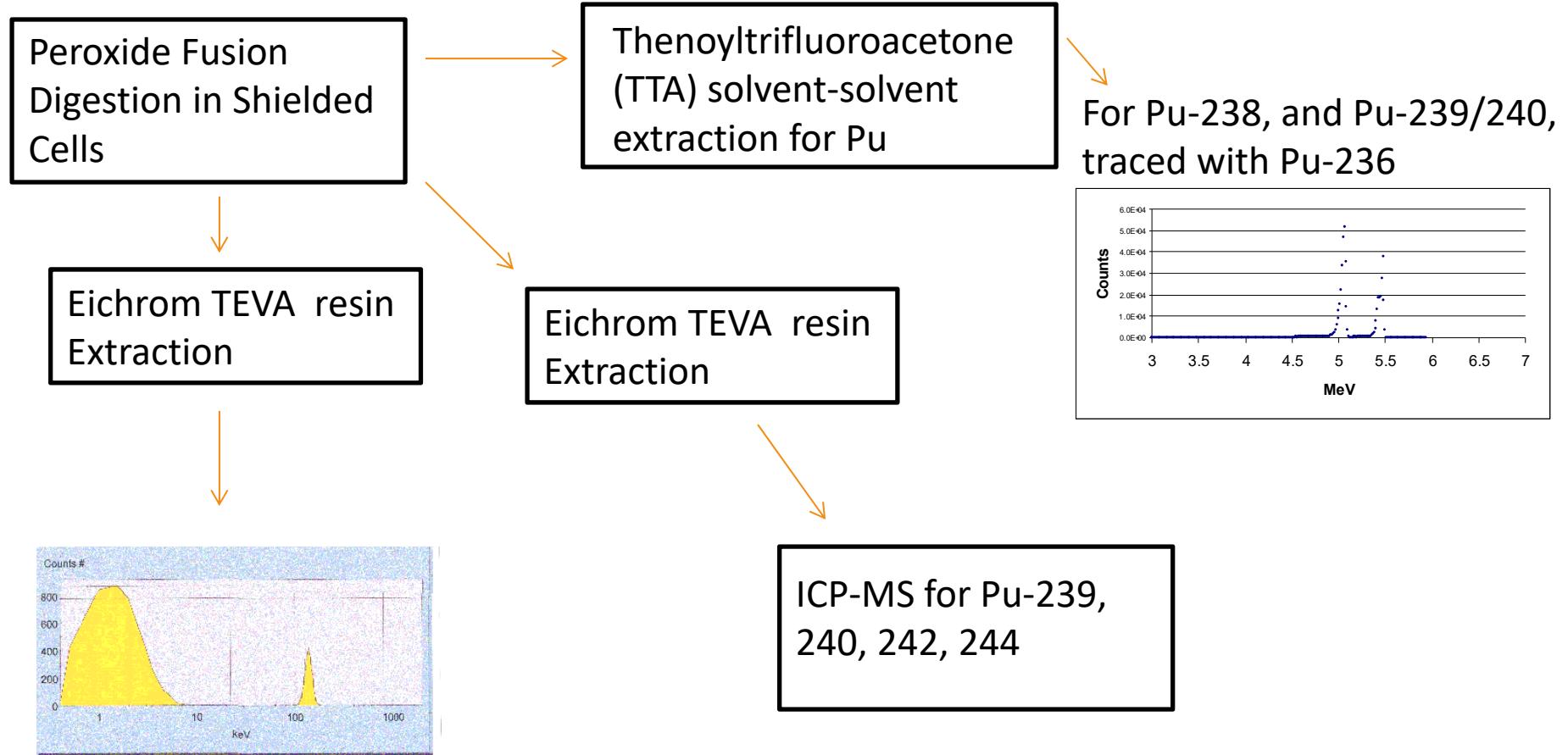
Neutron
Activation
Analysis to
determine
Sr Carrier
yields



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Pu-238, 239, 240, 241, 242, 244



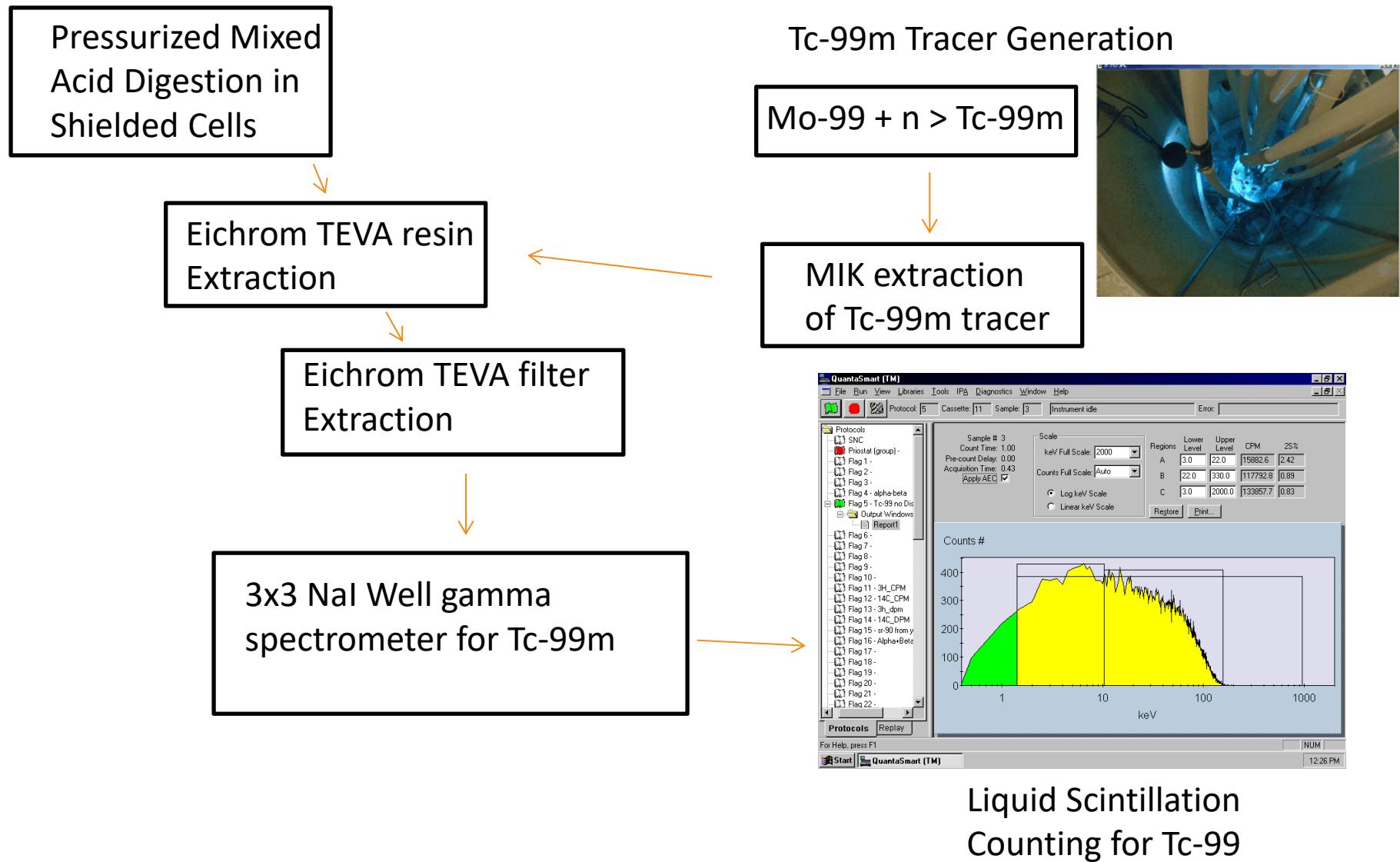
Liquid Scintillation Counting
for Pu-241 to Pu Alpha ratio



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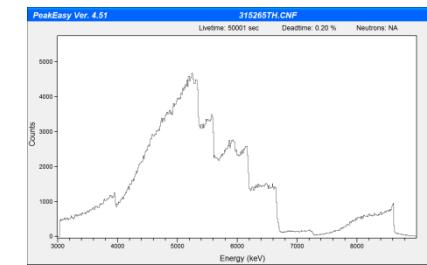
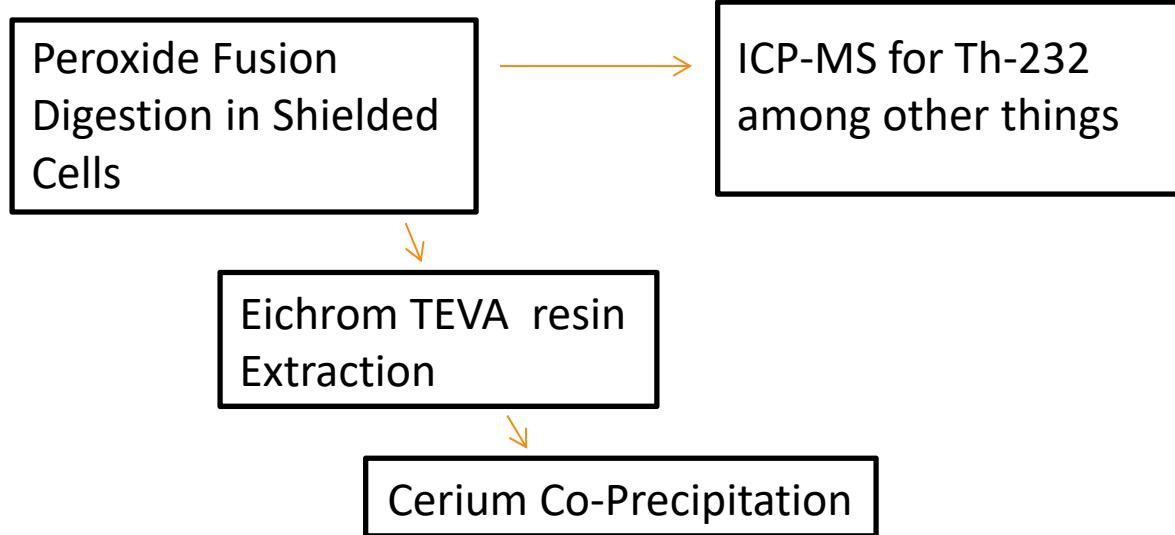
Tc-99



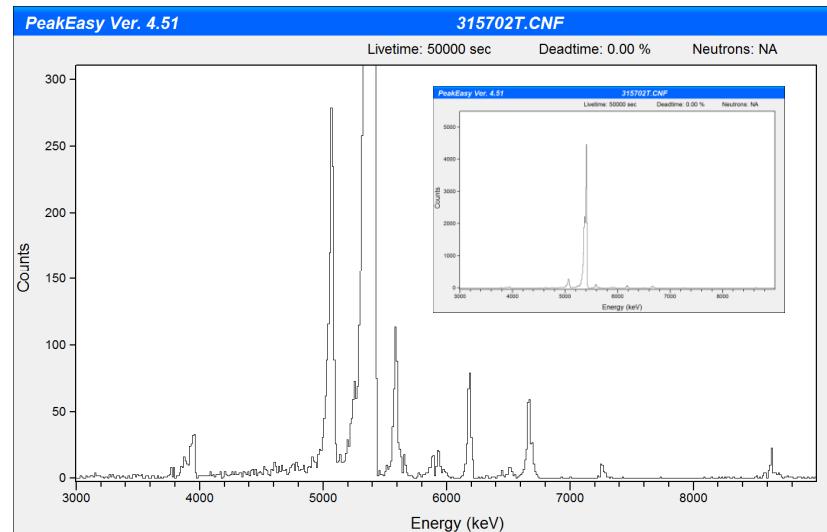
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Th-229, Th-230, Ac-227



Occasionally unexpectedly high in Thorium



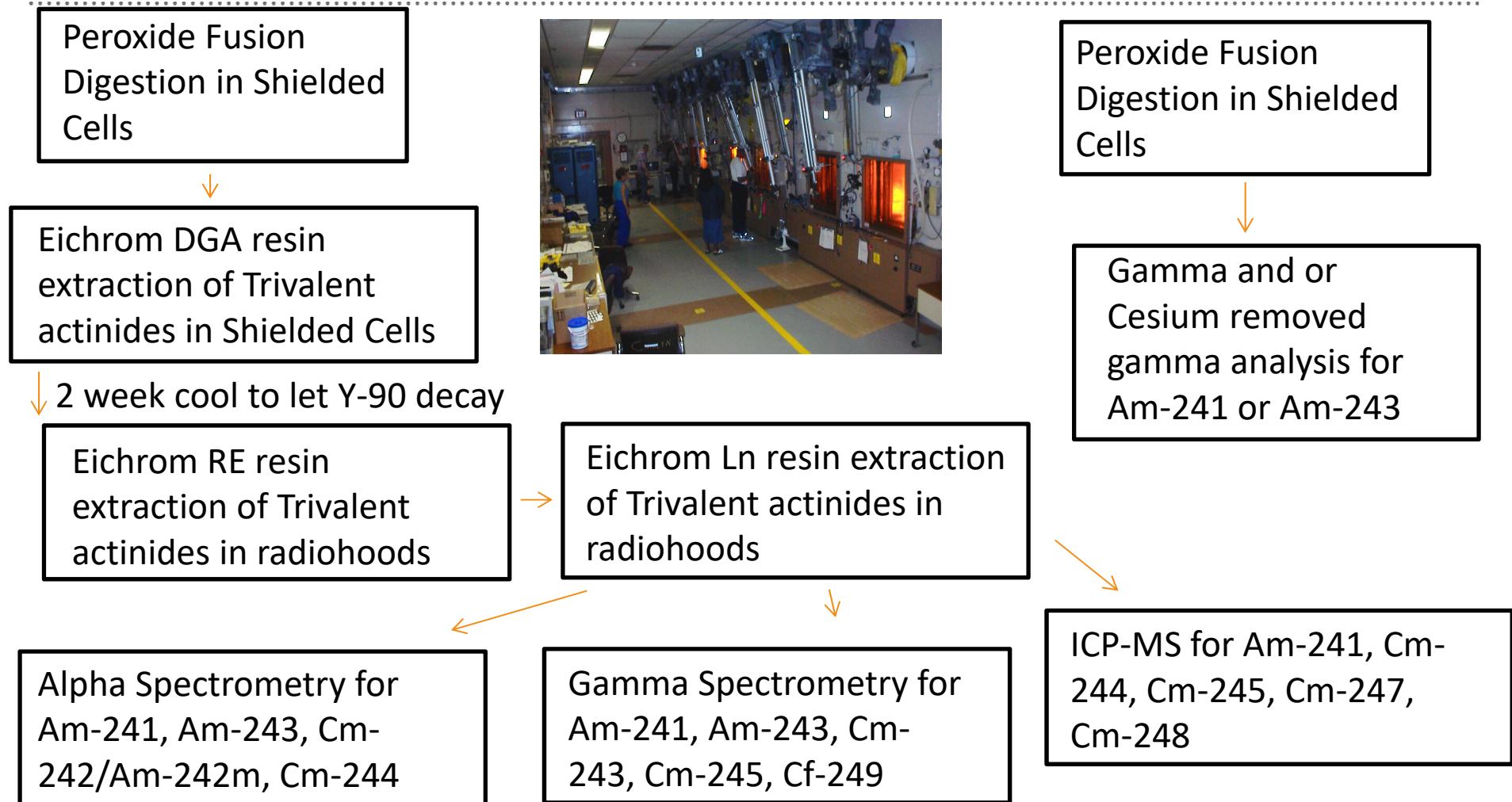
Alpha Spectrometry for Th-227, 229 and 230



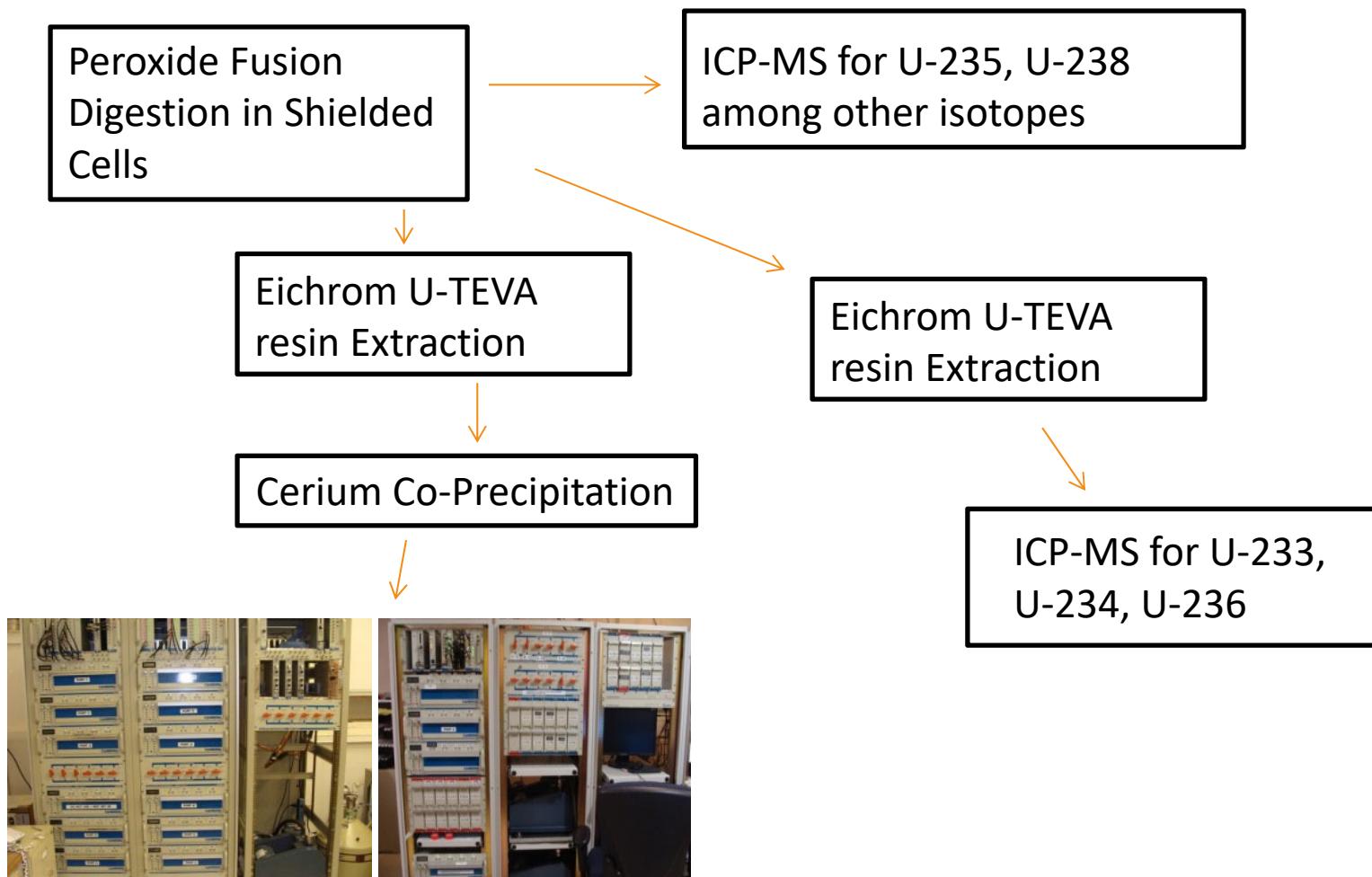
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Am-241, 242m, 243, Cm-242, 243, 244, 245, 248, Cf-249



U-232, U-233, U-234, U-235, U-236, U-238



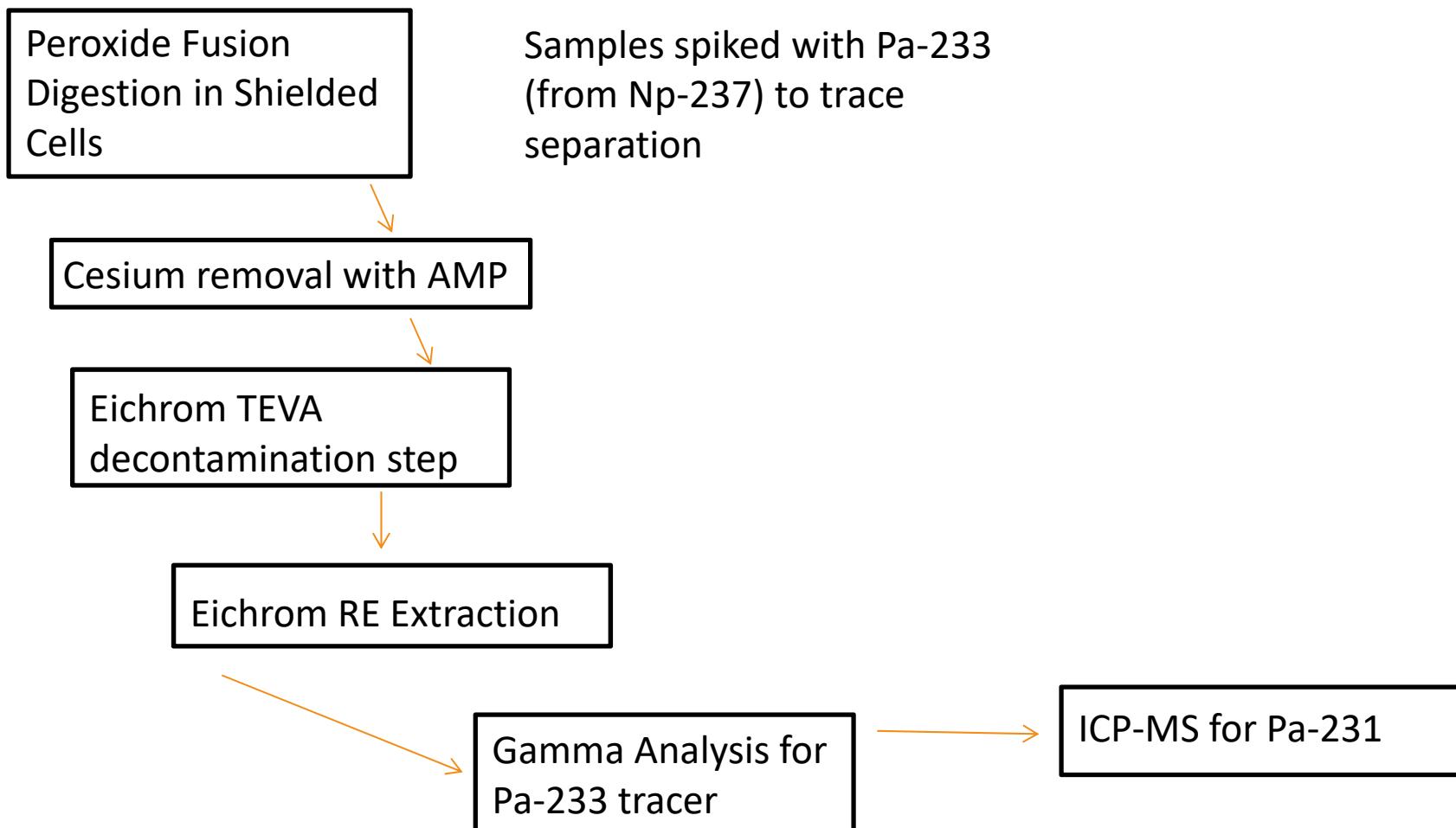
Alpha Spectrometry for U-232, traced with U-233 or ratioed to U-238 measured by ICP-MS



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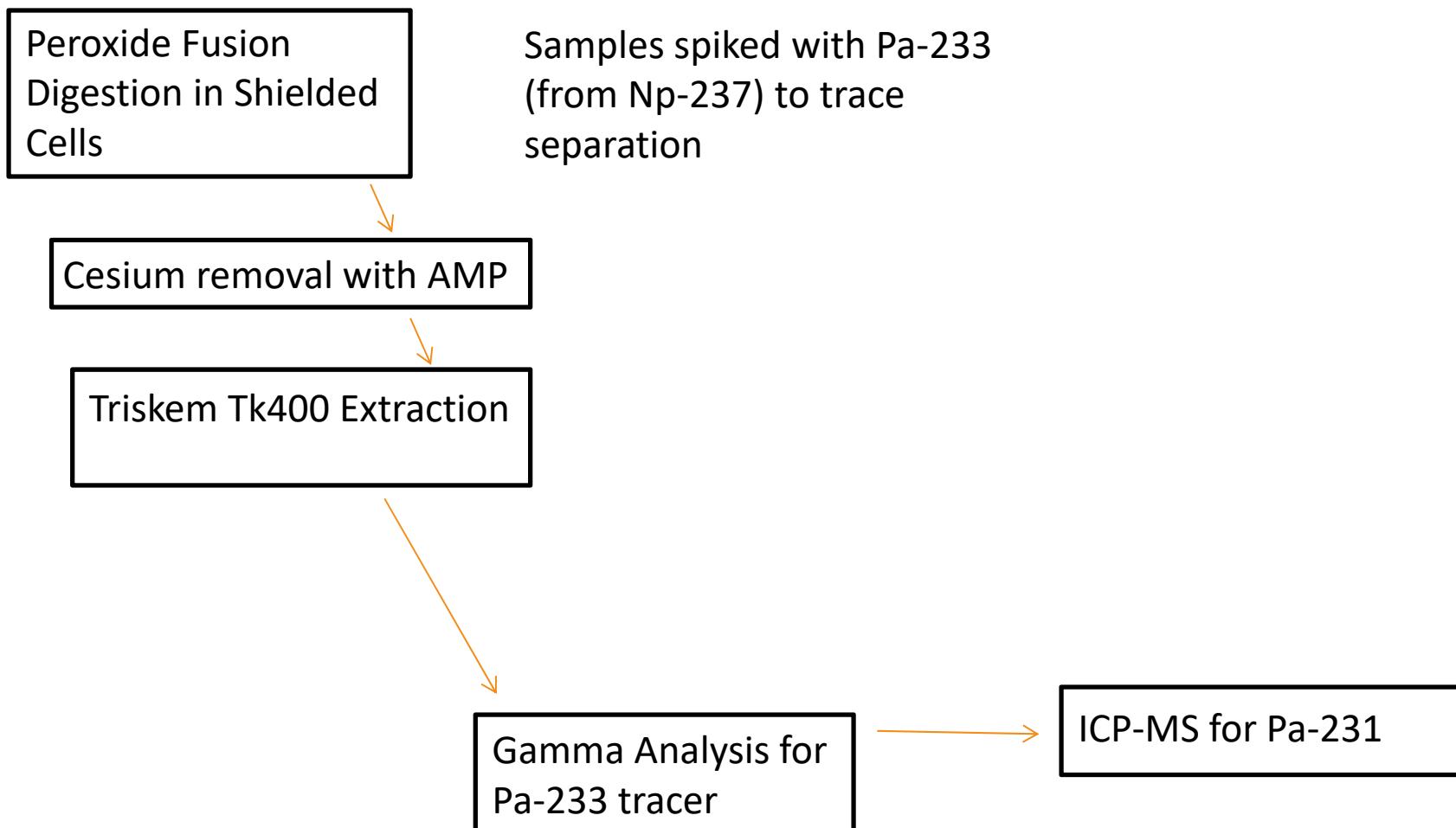
Pa-231 – Old Method



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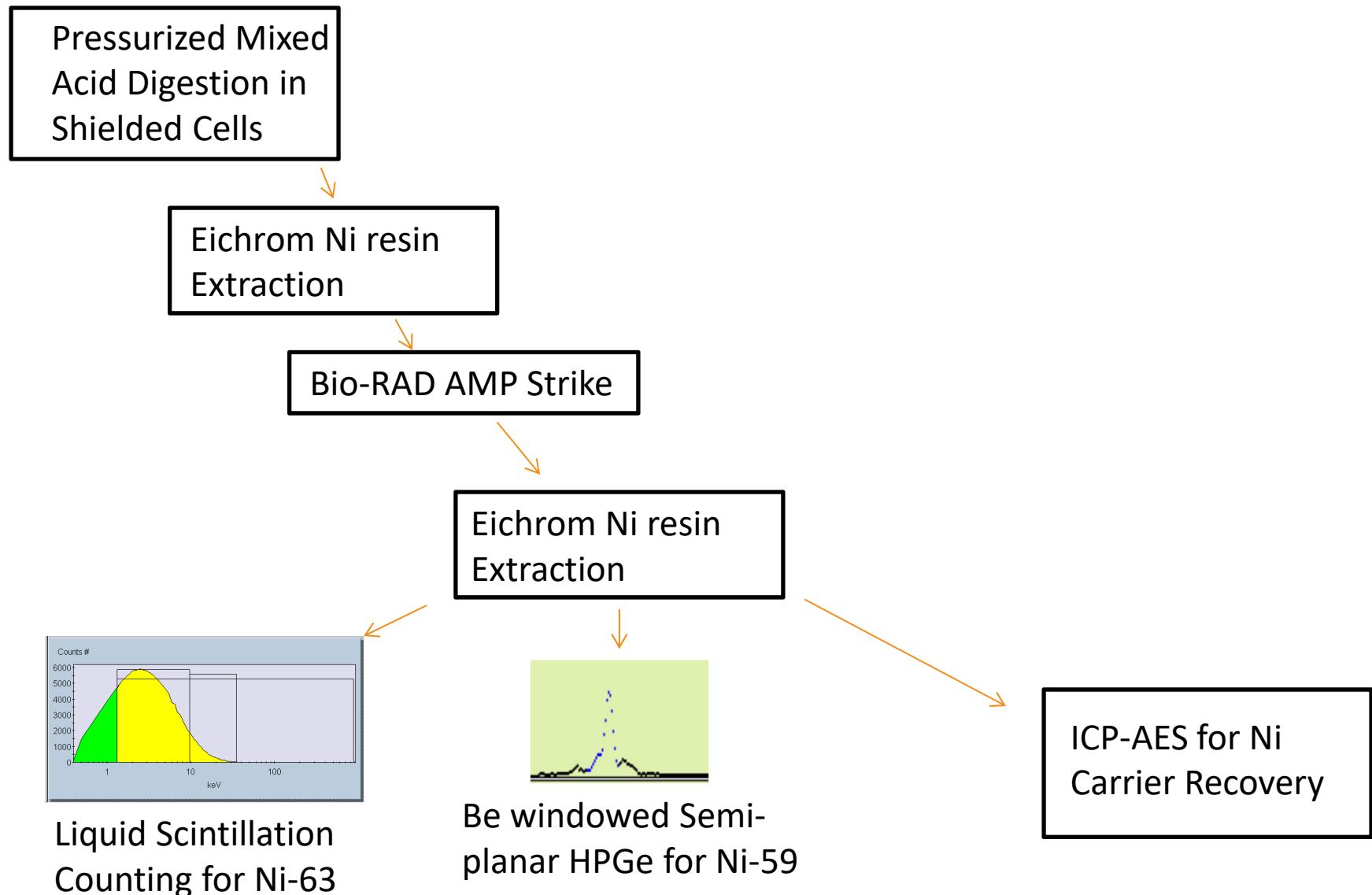
Pa-231 – New Method



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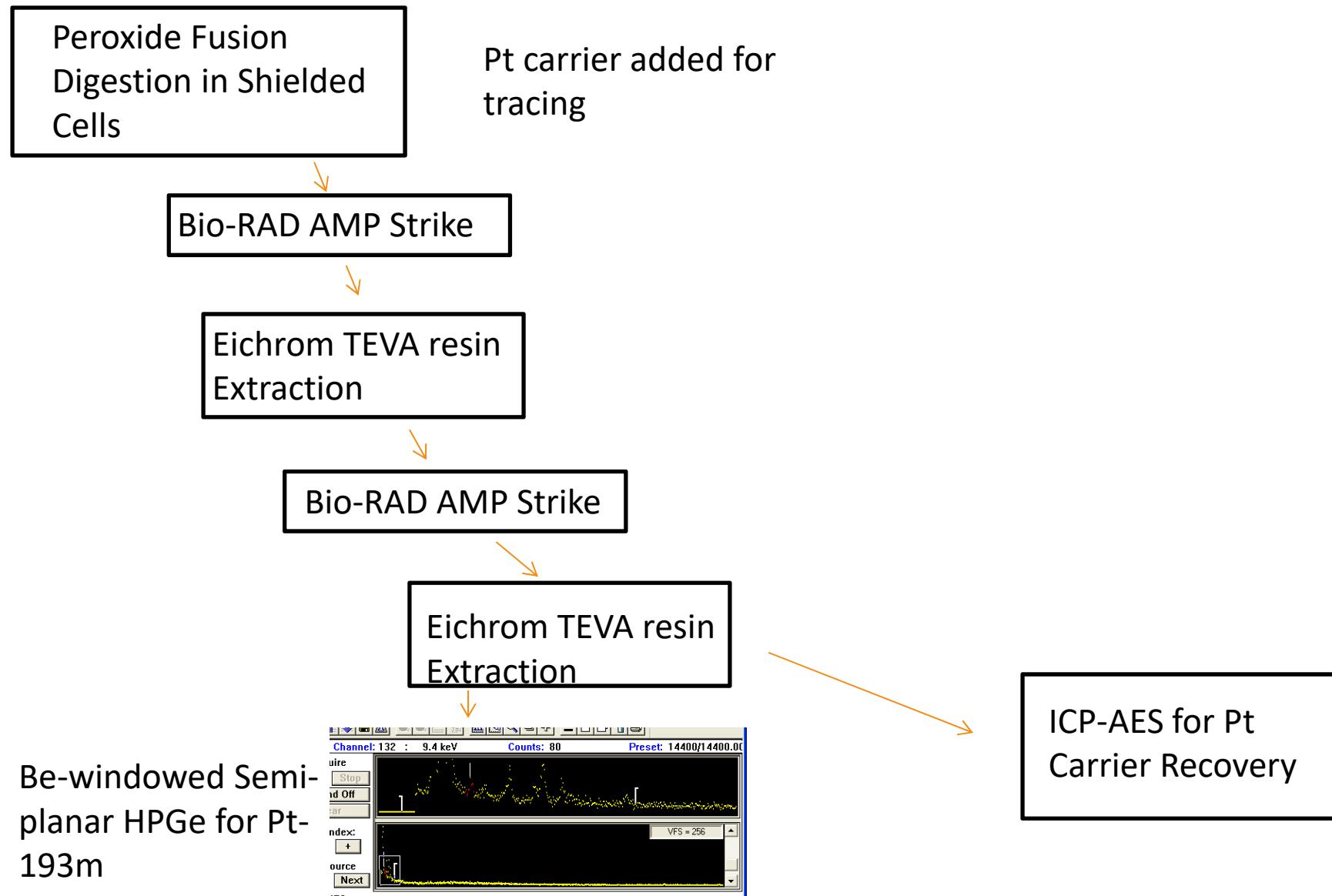
Ni-59/63



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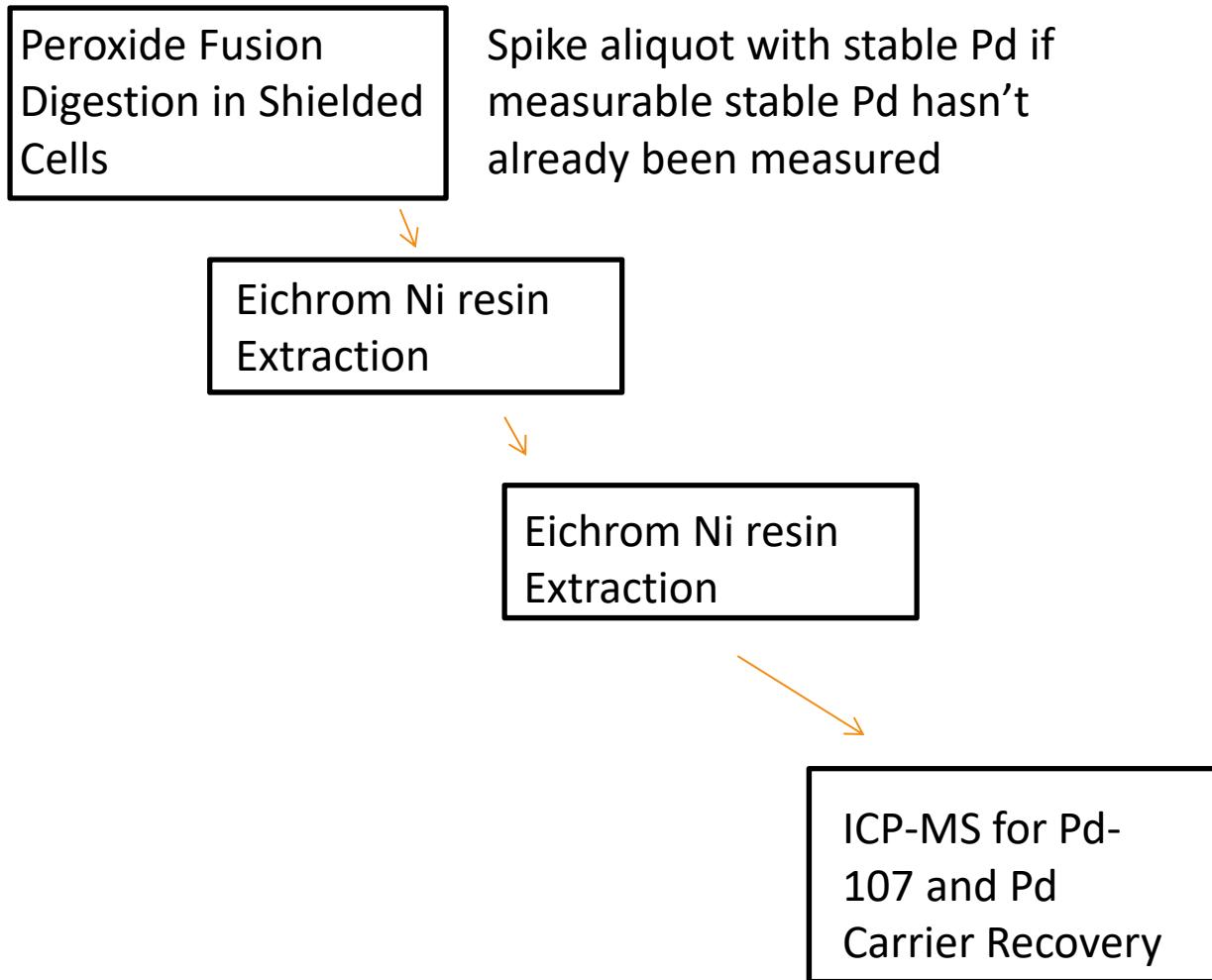
Pt-193m



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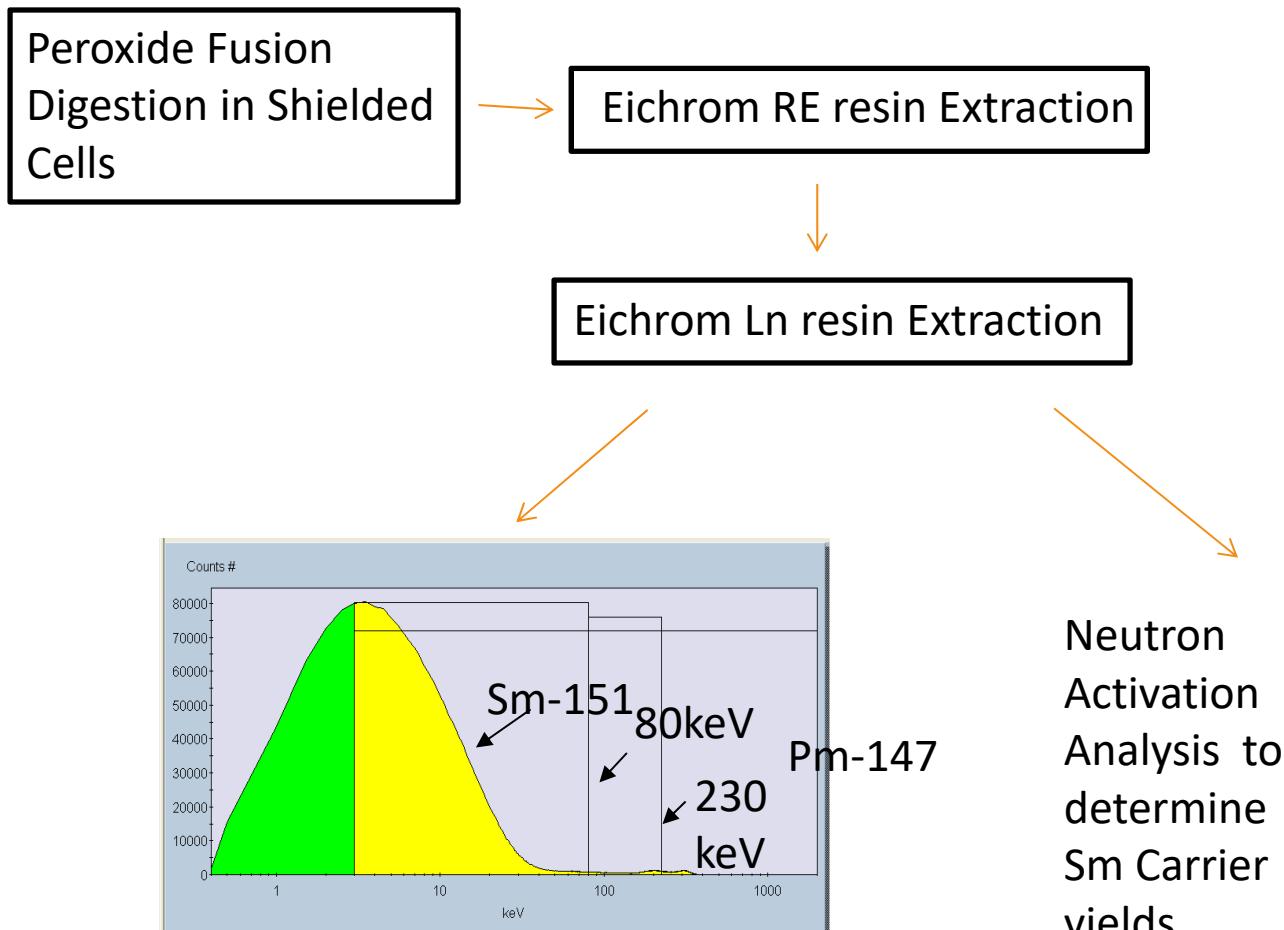
Pd-107



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Pm-147/Sm-151



Liquid Scintillation Counting for
Sm-151 and Pm-147

Neutron
Activation
Analysis to
determine
Sm Carrier
yields



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Acknowledgments

I would like to acknowledge the members of the Nuclear Measurement Team

- Robin Young
- Ceci DiPrete
- Gina Robbins
- Amanda Sadler
- Viet Nguyen
- Brooke Shore
- Kallee Fenker
- Tim Aucott
- Alex Brand
- Travis Deason
- Alejandra Hernandez-Jimenez

